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TECHNICAL DIVISIONS
GENERAL ELECTRIC COMPANY
RICHLAND, WASHINGTON

HW-11988

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May 1973

By *L. Pope 10/25/73*

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December 31, 1948

Atomic Energy Commission
Hanford Operations Office
Richland, Washington

Attention: Mr. R. C. Hageman
Chief, Operations Division

Gentlemen:

BEST AVAILABLE COPY

METAL RECOVERY PROCESSES SURVEY

APPROVED FOR
PUBLIC RELEASE

6/18/97

Those processes which will yield decontaminated uranium by means of solvent extraction or uranyl ammonium phosphate-fluorination are currently being studied by the General Electric Company in various combinations, in order to determine the most attractive processes from the standpoint of construction cost, time to begin operation, operating cost, engineering design, and process chemistry.

We submit for your information the attached diagram and tables which indicate our present approach toward obtaining the information we need for basing sound recommendations. The present summary is representative of our progress to date. Additional information is expected to result in some revisions.

By March 1, 1949, we expect to have completed the compilation and survey of the data, including estimated construction costs.

In the next few weeks, our activities will be chiefly concerned with the development of construction costs.

The following are the cases or proposals we are studying and comprise the combinations of solvent extraction, bismuth phosphate (Bi PO_4 , = present process) and uranyl ammonium phosphate processes to yield uranium hexafluoride as the end product:

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Proposal or Case No.	Process or Combination	Plants made Obsolete	Future Plants in Operation
1.	Bi PO ₄ plus uranyl ammonium phosphate (U.A.P.) for two year old waste	None	Bi PO ₄ plus U.A.P. (at less than maximum capacity)
2.	Bi PO ₄ plus Solvent Extraction (S.E.) for two year old waste	None	Bi PO ₄ plus S.E. (at less than maximum capacity)
3.	Bi PO ₄ plus U.A.P. for aged and current waste	None	Same as 1.
4.	Bi PO ₄ plus S.E. for aged and current waste	None	Same as 2.
5.	Redox plus U.A.P. for old waste	Bi PO ₄ and U.A.P.	Redox
6.	Redox plus S.E. for old waste	Bi PO ₄ and S.E.	Redox
7.	Redox for old and current metal waste	Bi PO ₄	Redox at less than maximum capacity
8.	Solvent extraction to separate Pu from U; Pu processed as at present via Bi PO ₄ . U decontaminated via S.E. plus separate S.E. plant for aged waste	S.E. for aged waste	S.E. for separating Pu from U and decontamination of U; Bi PO ₄ for Pu decontamination
9.	Solvent extraction to separate Pu from U in current waste; Pu decontaminated via Bi PO ₄ , U decontaminated via solvent extraction	None	S.E. for separating Pu from U and decontaminating U operated at less than maximum capacity plus Bi PO ₄ for Pu

We recognize that work has been in progress on other processes for metal recovery -- for example, a straight fluorination process and the electrolytic process. It is our opinion, however, that the U.A.P. and solvent extraction processes are further developed and that design work on either could proceed on the basis of available information. Data may be available shortly on other processes which may lead to the decision to consider these processes and combinations also.

The essential bases we are using in this study are as follows:

1. The nine proposals above are being compared at capacities which will yield four metric tons of decontaminated metal per day from stored and current waste, until the stored waste is totally recovered or reaches the constant inventory as required by Proposals 1 and 2.

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2. All proposals are to yield the hexafluoride as the end product. It is recognized that all steps required to yield this product need not necessarily be conducted at this site, except in those cases wherein fluorination is a required step for decontamination. (Proposals 1, 3, and 5) Unit costs and capacities of other manufacturing locations will be used where applicable in our study.
 3. Consideration is not being given at this time to the details of process revisions which may also yield recovered plutonium from metal wastes. For Proposals 7 and 9, however, a dotted line in the diagram indicates the path of any such desired recovery.
 4. All solvent extraction processes are expected to yield decontaminated uranium nitrate solution.
 5. For solvent extraction processes, waste metal preparation for feed will be by the U.A.P. - caustic metathesis process.
 6. The sluicing method as proposed by both Carbide and Kellex will be used for removal of waste from storage tanks. Kellex construction cost figures for this step will be used where applicable.
 7. The U.A.P. processes will be based on two U.A.P. precipitations. Carbide has provided data to show the satisfactory decontamination obtained by this departure from the proposal of their "Status Report".
 8. Solvent extraction processing will vary as follows:
 - a. For processes in which column feed is prepared directly from current metal (Proposals 5, 6, 7, 8, 9), three uranium cycles will be used.
 - b. For processes in which U.A.P. - Metathesis is used to prepare feed from Bi PO_4 process current and aged waste (Proposals 2, 4, 6, 7, and 9), two uranium cycles will be used.
 - c. For Redox processes, the third cycle waste will be "cribbed".
 - d. For processes utilizing U.A.P. - Metathesis, second cycle wastes in the solvent extraction step will be "cribbed".
- We recognize that "cribbing" of any activity may have to be abandoned. Such a policy will, however, apply to any present or future process considerations.
9. All preparation of the tetrafluoride is to be done via dry methods. Carbide has shown satisfactory decontamination by this means as a step in their U.A.P. processes yielding the hexafluoride as the final product.
 10. A 99+ % yield is assumed for all processes.

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11. For neutralized process wastes which can be concentrated, consideration will be given to concentrating in the plant buildings to a crystallization temperature of 20° C. Further concentration may be considered at the point of final storage.
12. Existing spare or idle facilities (U - Canyon) will be used wherever economically possible in the proposed processes.
13. Construction cost estimates will be made, utilizing in the best possible manner cost data now available on presently proposed construction (Redox). It may also be necessary to bring the cost of existing types of construction to present-day figures in some cases, for the comparison.
14. Value of existing facilities which may be utilized will not be included in construction cost figures. Cost of adaptation will, of course, be added into construction costs.
15. For those proposals in which a plant is built separately for old waste recovery and which is obsolete when the recovery program is completed, we propose a single plant to process waste from both areas (200-E and 200-W).
16. For permanent operations, complete facilities will be provided in both areas. This requirement may be limited to processing of active material; economics should favor centralized processing of decontaminated products.
17. The degree of overdesign incorporated in our study will be essentially the same as for the present Redox design. This includes spare cells and a 20% process capacity safety factor.
18. Remote control and remote maintenance will be proposed in all processing wherein shielding and contamination are involved.
19. The operating year is to be 365 days.
20. Chemical cost figures have been obtained from the Purchasing Division. Fluorine was taken at \$0.50/lb, the figure used by Carbide in its "Status Report".

The attached Block Flow Diagram (H-2-1520) shows, via strip comparison, the essential features of each proposal, together with indicated obsolescence.

The "Comparison of Daily Chemical Costs" is derived from the companion chart, "Comparison of Daily Chemical Consumption and Costs - Unit Processes", which subdivides the consumption into the "units" of each proposal, such as U.A.P. - Metathesis, Solvent Extraction, Fluorination, etc. Costs are shown for the period of metal recovery, as well as on an annual basis after the recovery program has been completed.

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The "Comparison of Storage of Neutralized Stored Wastes" shows the cumulative total gallons of waste to be stored over the period of aged metal waste recovery and also the annual storage after the recovery program has been completed. Costs of storage facilities at forty cents per gallon are also shown.

Very truly yours,

R H Beaton

R. H. Beaton, Head
Separations Technology Division

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- 1) All unit processes are counted at a production rate of one metric ton of waste (TW) per day.
- 2) All unit processes leading to decontaminated residues (DR) of K7 and P2 received for conversion to W6. The DRs are then used for the same unit processes. Others included the decontaminated residues (DR) of W6, P2 and W7.
- 3) Unit processes for waste plant cut-offs were available, otherwise from vendors' quotations. Waste prices included land-filling costs, or estimates.

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EXHIBIT OF DAILY PROCESS COSTS

Notes: 1) All processes are compared at a daily production rate of 1 metric ton of decontaminated waste. Current stock water is processed at a rate of 2 tons per day, and stored water is dechlorinated at a rate of 2 tons per day until an inventory of 2 years is reached in proposals 1 and 2, and until all stored waste is gone in other proposals.
2) All processes include costs of 7% and 7% for conversion of the final product to UF₄. DWP costs include the cost of 7% and 7% for conversion of the final product to UF₄.
3) Costs shown are based on the accompanying unit process costs by the formula shown at the column heads, and rounded to the nearest dollar.
4) Operating year 365 days.

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Material	1	2	3	4	5	6	7	8	9
NaOH	2,410	2,193	1,924	1,508	1,354	1,141	1,141	1,304	1,304
H ₂ O	1,272	838	1,272	838	1,021	784	784	838	838
Na ₂ CO ₃	544	991	435	842	545	673	673	956	956
Na ₂ SO ₄	435	160	435	160	275	91	91	42	42
Fe	775	775	775	775	775	775	775	775	775
H ₂ SO ₄	638	431	435	435	6	6	6	300	300
Na ₂ SO ₄	22	22	22	22	22	22	22	22	22
Na ₂ CO ₃	22	22	22	22	22	22	22	22	22
Na ₂ SO ₄	51	51	51	51	51	51	51	51	51
Na ₂ CO ₃	182	182	182	182	182	182	182	182	182
(Na ₂ CO ₃) ₂	182	182	182	182	182	182	182	182	182
Na ₂ SO ₄	34	34	34	34	34	34	34	34	34
Na ₂ CO ₃	23	23	23	23	23	23	23	23	23
Na ₂ SO ₄	2	2	2	2	2	2	2	2	2
Fe (Na ₂ CO ₃) ₂	91	91	91	91	91	91	91	91	91
Na ₂ CO ₃	34	34	34	34	34	34	34	34	34
H ₂ O	11	11	11	11	11	11	11	11	11
Ca (Na ₂ CO ₃) ₂	43	43	43	43	43	43	43	43	43
Zn carbonate	1	1	1	1	1	1	1	1	1
Ca (Na ₂ CO ₃) ₂	2	2	2	2	2	2	2	2	2
Na ₂ CO ₃	17	17	17	17	17	17	17	17	17
Na ₂ SO ₄	1,142	1,142	1,142	1,142	1,142	1,142	1,142	1,142	1,142
H ₂ O	0	0	0	0	0	0	0	0	0
Ca (Na ₂ CO ₃) ₂	2,700	2,700	2,700	2,700	2,700	2,700	2,700	2,700	2,700
Fe sulfamate	131	131	131	131	131	131	131	131	131
Revenue	842	842	842	842	842	842	842	842	842
Total Daily Cost, 1952-53	8,660	10,800	2,025	2,025	2,025	2,025	2,025	2,025	2,025
Annual cost, 1952-53	3,139,000	3,940,000	738,000	738,000	738,000	738,000	738,000	738,000	738,000
Sum of Cost, 1952-56 Incl.	22,655,000	27,781,000	22,308,000	27,650,000	27,650,000	27,650,000	27,650,000	27,650,000	27,650,000
Annual Cost, 1957 and After	2,462,000	2,462,000	1,897,000	2,105,000	2,105,000	2,105,000	2,105,000	2,105,000	2,105,000

* Costs may be reduced by inclusion of H₂O, process volume reduction.

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- 1) Decalcinated uranium produced at rate of four semis per day.
- 2) Irradiated fuel to be processed at rate of two metric tons monthly.
- 3) Decontaminated uranium produced at 600 g/hr except in cases 1, 3, and 5 where fluorination to UO₂ is required for adequate decontamination. Volumes of waste produced by fluorination tests indicated in parentheses.
- 4) Cost of starting waste solution volume of \$400 per gallon. Slurry waste requires additional steel tankage (all unit processes).
- 5) SLOD process waste volumes limited second cycle decalcination wastes now cribbed.
- 6) Condensate volumes estimated for saturation at 25°C.
- 7) Decontaminated uranium processing rates assumed to be cribbed. (Total without activity per ton U processed comparable with present 224, 946, values.)

[illegible]

Year 1998 not shown since only part the operations are required for proposals 3 & 5. All others would be at 1999 operating rate.

The SL-10 coupling process, producing a small volume Pu feed to the P-3 decontamination, would probably permit reduction of the waste volume from the phosphate process. Of the 5.125 million gallons annual waste shown, 3,022 million gallons are from the phosphate decontamination system. 45

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